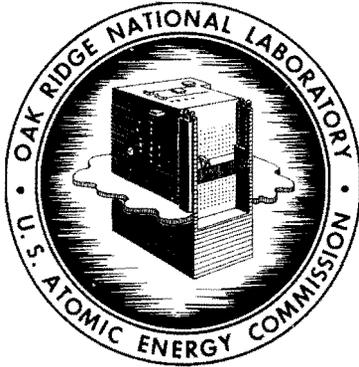


ORNL  
MASTER COPY



**OAK RIDGE NATIONAL LABORATORY**

operated by

**UNION CARBIDE CORPORATION**

for the

**U.S. ATOMIC ENERGY COMMISSION**



ORNL - TM - 354

COPY NO. - 4

DATE - September 17, 1962

USE OF THE DRY MAINTENANCE FACILITY  
FOR HRT MAINTENANCE

J. Paul Jarvis

ABSTRACT

A portable shield, described in ORNL CF 60-10-85, was utilized in performing nine semi-direct dry maintenance jobs on the HRT. The shield proved to be completely reliable, affording a considerable savings in maintenance costs and reactor downtime.

NOTICE

This document contains information of a preliminary nature and was prepared primarily for internal use at the Oak Ridge National Laboratory. It is subject to revision or correction and therefore does not represent a final report. The information is not to be abstracted, reprinted or otherwise given public dissemination without the approval of the ORNL patent branch, Legal and Information Control Department.

LEGAL NOTICE

This report was prepared as an account of Government sponsored work. Neither the United States, nor the Commission, nor any person acting on behalf of the Commission:

- A. Makes any warranty or representation, expressed or implied, with respect to the accuracy, completeness, or usefulness of the information contained in this report, or that the use of any information, apparatus, method, or process disclosed in this report may not infringe privately owned rights; or
- B. Assumes any liabilities with respect to the use of, or for damages resulting from the use of any information, apparatus, method, or process disclosed in this report.

As used in the above, "person acting on behalf of the Commission" includes any employee or contractor of the Commission, or employee of such contractor, to the extent that such employee or contractor of the Commission, or employee of such contractor prepares, disseminates, or provides access to, any information pursuant to his employment or contract with the Commission, or his employment with such contractor.

## INTRODUCTION AND HISTORY

Fabrication of the dry maintenance shield\* was completed in the ORNL shops early in September, 1960. The shield was assembled at the fabrication shop and operational checks were performed. A few minor modifications were made to the shield under the direction of P. P. Holz. The shield was delivered to the 7500 area the same month.

Early in October, 1960, the shield was assembled and tested in conjunction with a program to familiarize and train personnel in the use of the shield. Detailed job procedures were written covering the use of the shield in performing some of the simpler maintenance jobs on the reactor system.

During the period October 1, 1960 through March 11, 1961, the shield was utilized on nine jobs to a very definite advantage in the HRT maintenance program. A summary of these nine operations is listed in Table 1.

## LIMITATIONS OF THE SHIELD

### Radiation Levels

The shield was utilized to shield personnel from radiation levels of from 10 r/hr. to over 200 r/hr. Table 2 was compiled from actual radiation measurements and personnel exposure records during the performance of nine jobs.

### Tools

The standard maintenance tools were used during this maintenance work. A special off-set lifting hook was built for use in replacing the blanket dump valve (PCV-252), allowing the job to be accomplished through the one lower plug opening.

Lead shot bags were used for shielding around tool handles. This proved to be adequate, but it would be more convenient to have split lead plugs to fit between the shield tool openings and the tool handles. There would also be an advantage in a standard tool handle diameter.

As tools were withdrawn from the shield, they were wiped down with a wet cloth. There was no significant contamination of the shield top during this maintenance work.

### Visibility

Visibility was excellent through the shield as compared to working through water. The built-in lights proved quite adequate; however, for close viewing, a 500-w projector bulb on a drop cord was used to position the light source closer to the work area.

---

\*P. P. Holz, Dry Maintenance Facility for the HRT, ORNL CF-60-10-85 (Oct. 11, 1960).

Table 1. HRT Jobs Performed with the Aid of the Dry Maintenance Shield

Operation	Date	Days After Run Number		Radiation Level		Duration of Job	Average Daily Radiation Exposure	Comments
		Days	Run No.	Lower Plug Removed	Shield in Place			
(1) Replace multicclone filter.	10/17/60	2	22	10 r/hr at elev. 830'	General background of 5 mr/hr	8 hr	30 mr	Job could not be done flooded because there was no way to keep cell water out of the reactor system.
(2) Smear cell floor to locate leak in high-pressure system.	12/6/60 and 12/10/60	2	23	120 r/hr at elev. 830'	General background of 20 mr/hr	120 hr	50 mr	Leak found at E. head fuel feed pump. This smear method of leak hunting proved quite effective. The cell could not be flooded.
(3) Replace E. head fuel feed pump.	12/10/60	6	23	100 r/hr at elev. 830'	General background of 20 mr/hr	10 hr	1.5 r	The high personnel exposure was accumulated during the transfer of the pump to the storage pool. The shield was used to advantage during the actual replacement of the pump.
(4) Smear cell floor to locate leak in reactor system.	2/10/61	2	24	From 120 r/hr to Over 200 r/hr	20 mr/hr 80 mr/hr	168 hr	70 mr	Leak found in line 107 at fuel dump tanks. The cell could not be flooded during this type of leak hunting. Maintenance shield saved weeks of shutdown time. Personnel exposures were very low as a result of this shield.
(5) Install a clamp-on freezer on LCV-145 fuel letdown valve.	2/17/61	9	24	200 r/hr	80 mr/hr	10 hr	85 mr	Freezer installed to isolate the reactor system from the fuel letdown heat exchanger as part of the letdown heat exchanger replacement. As a result of these freezers the reactor system was not contaminated with cell water during this maintenance period.

Table 1. (continued)

Operation	Date	Days After Run Number		Radiation Level		Duration of Job	Average Daily Radiation Exposure	Comments
		Days	Run No.	Lower Plug Removed	Shield in Place			
(6) Install a clamp-on freezer on line 113 at gas separator.	2/18/61	10	24	50 r/hr	10 mr/hr	12 hr	35 mr	Freezer installed to isolate the reactor system from the fuel letdown heat exchanger as part of the letdown heat exchanger replacement. As a result of these freezers, the reactor system was not contaminated with cell water during this maintenance period.
(7) Replace PCV-252 blanket dump valve.	2/23/61	15	24	10 r/hr	15 mr/hr	10 hr	20 mr	This was a difficult job due to the location of the valve. It is doubtful that the same job could have been done if the radiation level had exceeded 50 r/hr with the roof plug out.
(8) Umbolt flanges 111, 112, and 113 at the fuel letdown heat exchanger. Saw off heat exchanger flanges and blank off lines	2/24/61 and 2/25/61	16	24	50 r/hr	20 mr/hr	32 hr	40 mr	Cell was flooded to elev. 820' and flanges unbolted dry. The flanges were blanked off, keeping cell water out of the reactor system during the removal of the fuel letdown heat exchanger.
(9) Replace HCV-141 and HCV-142 chemical plant inlet and outlet valves.	3/6/61 and 3/11/61	26	24	10 r/hr	15 mr/hr	144 hr	35 mr	On this job, the eccentric module was used with the lead bridge. This allowed the two lower plugs to be removed.

Table 2. Radiation Limitations

Radiation* Level With Bottom Roof Plug Removed	Radiation Level Through the 10.5" Steel Shield	General Background 36" Above the Shield. Avg. No. of Tools Through the Shield.	Average Working Time for a 60-mr Exposure	Job Limitations
(r/hr)	(mr/hr)	(mr/hr)	(hr)	
200	100	80	4	(1)
200	20	60	5	(1)
120	15	30	9	(1),(2)
50	---	10	16	(1),(2),(3)
10	---	5	24	(1),(2),(3)

\*Two to ten days after reactor shutdown.

NOTE: In all cases additional shielding of lead shot and brick was used to cover cracks in the shield. This accounts for the variations in radiation backgrounds and working times for a given radiation level.

- (1) Tighten flanges, smear cell floor and observe equipment, etc., through the shield.
- (2) Replace valves in a vertical position, replace feed pumps and purge pumps.
- (3) Install clamp-on freezers, replace valves in horizontal position; limiting factor was the size of the opening through the shield.

There is a need for larger viewing windows. Two 4-in.-diam. lead glass windows were used in existing tool holes. These viewing windows are too small to allow the tool operator to see through. It was necessary for a second man to observe and direct the operation. A 12-inch by 12-inch lead glass viewing window mounted in the main slide as close to the eccentric module as possible would allow the tool operator to observe the work. An 8-power monocular was used to great advantage in viewing through the lead glass windows.

#### Movement of Tools and Equipment

The movement of tools and equipment is restricted as compared to underwater maintenance. This restriction did not prove to be a great problem in any of the nine jobs. Tool flexibility could be increased if ball-joint-type tool shield plugs were used in place of the straight tool holes. This same flexibility was gained more crudely by utilizing a 4-inch hole for a 1-1/2-inch tool handle and shielding with bags of lead shot.

The positioning of equipment in the cell was no problem. Off-set lifting hooks were built to further aid in the movement of equipment.

#### Mobility

The shield was set up over ten different work areas a total of twenty times. The first ten times required about 3-1/2 hours per set up. This average set-up time was reduced to about 2 hours during the last ten times.

#### CONCLUSIONS

The use of the dry maintenance shield eliminated the need for flooding the reactor cell for shielding. This results in the following savings in reactor downtime:

1. One day to flood reactor cell.
2. Two days to drain reactor cell and dry out cell.
3. Two days to remove light water from the reactor system, on occasions when light water leaks past the freeze plugs.
4. It eliminates the need of disposing of 180,000 gal of cell water that usually contains from 5 to 10 curies of activity.

The use of the dry maintenance shield increased personnel radiation exposures about 20% as compared to underwater maintenance.

This increase in personnel exposure can be eliminated with the following modifications to the shield:

1. Replace the tool holes with split ball shield plugs. This will provide the needed flexibility for tools and shielding around tool handles.

2. Provide bolt-on-type lead shielding over the eccentric module gear drive and ring.
3. Install a 12-in. x 12-in. lead glass viewing window in the main shield as near as possible to the eccentric module. Install a 12-in. x 12-in. lead glass viewing window in one of the 2-ft 6-in. long modules.

The use of personal radiation monitors by personnel working through the shield will result in lower personnel exposures.

The time required to complete a specific job working through the dry maintenance shield was about the same as for underwater maintenance. The increase in visibility was an advantage that more than compensated for the restriction in tool and equipment movement.

Distribution

- 1-3. DTIE, AEC
4. M. J. Skinner

